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Zhiwei Hu^{a, e}, Qigui Yang^c, Jintong Wu^d, Duc Nguyen-Manh^b, M. Yu. Lavrentiev^b, Pierre Desgardin^a, Jérôme Joseph^a, Marie-France Barthe^{a, *}

^a CEMHTI, CNRS, UPR3079, University of Orléans, F-45071 Orléans, France

^b UK Atomic Energy Authority, CCFE, Culham Science Centre, Abingdon, Oxon, OX14 3DB, United Kingdom

^c Institute of High Energy Physics, CAS, 100049 Beijing, China

^d Department of Physics, Post-office box 43, FIN-00014 University of Helsinki, Finland

^e Institute of Radiation Physics, Helmholtz Center Dresden-Rossendorf, 01328 Dresden, Germany

Abstract

Microstructural evolution-driven degradation governs material properties and is closely linked with defect behavior. Quantitatively characterizing defects and their evolution is essential for elucidating the underlying degradation mechanisms. To this end, the defects were introduced at room temperature using self-ion irradiation for damage levels ranging from 0.01 to 2 displacements-per-atom. The depth distribution of vacancy defects were characterized by means of a variable-energy positron beam and compared with simulation results. Quantitative analyses of vacancy in the most damaged regions were carried out by combining positron annihilation spectroscopy with a simulated annealing algorithm parametrized with a positron trapping model and first-principles calculations. The defect size distribution -from single vacancy to vacancy clusters- was assessed at each damage levels, providing insights into the quantification of early-stage vacancy defects. Our results revealed that at 0.01 dpa, nearly all vacancy defect exists as isolated single vacancies. The proportion of isolate single vacancies gradually drops to ~20% with increasing damage level, reaching a steady state (> 0.5 dpa). Meanwhile, clusters consisting of four or more single vacancies account for ~12 % of the total vacancy defects. when the damage level exceeds 0.1 dpa, the formation of large clusters containing more than 15 vacancies, although limited to less than 1% in population, cannot be

excluded. Furthermore, the estimated vacancy accumulation trend is consistent with available computational results, and unveils that the vacancy clustering is more pronounced in molybdenum than in tungsten at the early stage evolution.

Keywords: Vacancy quantification, Defect distribution, Positron annihilation spectroscopy, First-principles calculation, Fusion materials,

1. Introduction

Future fusion power plants require high-performance materials. Tungsten (W) and Molybdenum (Mo) are the most auspicious candidates for plasma-facing components (PFC) [1,2] thanks to their high melting points, thermal conductivity, and low sputtering yield [3]. Compared to W, Mo offers advantages in certain properties and has been chosen to line the inner wall of the Experimental Advanced Superconducting Tokamak (EAST) [4]. For example, radiation-induced activation is lower in Mo than in W [5], and Mo is simpler to manufacture into lighter components than W components. In the operation of the future reactor, monitoring the stability of the plasma is essential. To achieve this, the plasma mirror (PM), as a part of the optical diagnostics system, is used to reflect the light from the plasmas [4]. Promising results [5–7] have been revealed using Mo as PM. Moreover, in tokamaks, inner walls and structural materials are subjected to hydrogen (H) and helium (He) plasma and neutron irradiation, which degrade their properties. Recent research conducted by Lavrentiev et al. [8] demonstrates that the reflectivity of Mo remains stable up to fluences of 10^{18} m^{-2} under self-ion irradiation. In this study, the irradiated Mo samples were characterized by Thermal desorption spectrometry after exposure to deuterium plasma. The deuterium inventory increased with the damage level until reaching a saturation, similar to findings in previous work on W [9]. According to modeling, this saturation occurs when the damage dose reaches 0.5 dpa [10]. This saturation reveals that a steady-state microstructure forms once the damage reaches a certain threshold level [9,11]. This is probably due to competition between Frenkel-Pairs (FPs) recombination, cascades

53 overlapping [12], and agglomeration of self-interstitial loops, accompanied by the shrinkage of
54 the vacancy clusters [13]. The resulting steady-state microstructure consists of vacancy defects,
55 including single vacancy (V_1) and vacancy clusters (V_n), dispersed within a network of
56 dislocations formed by self-interstitial atoms (SIAs). Both interstitial and vacancy defects
57 capture H and He isotopes, particularly the vacancy defects [14,15]. Furthermore, light-element
58 impurities (LEs) also influence defect evolution [16–19] and serve as potential traps for H [20].
59 The interaction of H and He with the defects has been observed to cause blistering [21–23],
60 bubbles [24,25], and fuzzy surfaces [17]. The trapping properties of H and He in vacancy
61 defects are a major parameter in the degradation of material properties [26–30], and the kinetic
62 appearance of bubbles and blisters depends on the nature of the vacancy [15,20,31,32].
63 Therefore, quantifying defects is a crucial step in predicting tritium retention, swelling and
64 material evolution in tokamaks.

65 Positron annihilation spectroscopy (PAS) is renowned for nondestructive technique to
66 characterize atomic-scale vacancy defects, thanks to its unique sensitivity. For instance,
67 Lhuillier et al. [33,34] and Debelle et al. [35] have identified the signal of the V_1 in W and
68 monitored its evolution with a post-irradiation isochronal annealing experiment. They have
69 observed the activation temperature for V_1 between 523 and 573 K [35]. Positron annihilation
70 is primarily implemented in two types of spectroscopy: Doppler broadening spectrometry and
71 positron annihilation lifetime spectroscopy (PALS), which allow the measurement of two
72 annihilation characteristics: the momentum distribution of annihilated positron-electron (e^+e^-)
73 pairs and lifetime, respectively [36]. Theoretically, each type of vacancy defect exhibits specific
74 annihilation characteristics, difficult to determine all of them through experimental methods.
75 For this issue, first-principles calculations can provide helpful information [37,38]. Recently,
76 Yang et al. [39] have shown the accurate calculation of the Doppler Broadening Spectrum (DBS)
77 and the lifetime of several transition metals using the two-component density functional theory

(TCDFT) developed by Makkonen et al. [40]. In tungsten, the evolution of annihilation characteristics for different types of vacancy defects from V_1 to the vacancy cluster has been revealed by calculation and experiment [41]. Considering the similarities between W and Mo, we used this first-principles method to calculate the DBS and the lifetime of the vacancy defects in Mo. This method enables the determination of the positron annihilation characteristics (lifetime, S , and W) for vacancy defects that are challenging to isolate and quantify experimentally. In line with the simulation, we employed Doppler Broadening spectrometry with a variable monoenergetic positron beam to characterize the radiation-induced defects in Mo using self-ion at Room Temperature (RT). Furthermore, we conducted the method recently proposed in [42] for qualifying the proportion of each type of vacancy defect from DBS by combining a simulated annealing algorithm, parametrized with a positron trapping model and first-principles calculations. The estimated vacancy defect distribution was compared to Object Kinetic Monte Carlo (OKMC) simulations [43].

2. Methods

2.1 Sample preparation and irradiations

Polycrystalline Mo samples were polished until a mirror-like surface and then annealed at 1200 °C for 2 h. The prepared Mo samples were damaged using 1 MeV self-ion at RT with various fluences, ranging from 2.1×10^{16} to $4.2 \times 10^{18} \text{ m}^{-2}$. The program Stopping and Range of Ions in Matter (SRIM) was used to compute the damage profile, with the ‘Kinchin-Pease quick calculation’ option recommended by Stoller et al. [44] and recently checked by Lin et al. [45]. The damaged region was predicted between 0 and 400 nm, with the peak at around 110 nm. [Tab.1](#) summarizes the irradiation conditions. 34 eV was used as the threshold displacement energy (TDE) [46]. The fluences used in this work induced maximum damage, quantified in displacement per atom (dpa), ranging from ~0.01 dpa up to ~2 dpa. Further details on preparation and irradiation have been reported in [8].

103 *Tab 1: irradiation conditions for 1 MeV-Mo in Mo using SRIM-2008 Kinchin-Pease quick calculation with a TDE of 34 eV*

Fluence (cm ⁻²)	2.1×10^{12}	2.1×10^{13}	6.3×10^{13}	1.1×10^{14}	2.1×10^{14}	4.2×10^{14}
Damage (dpa)	0.0096 (~0.01)	0.096 (~0.1)	0.29 (~0.30)	0.48 (~0.5)	0.96 (~1)	1.92 (~2)
Time (s)	64	143	927	2109	2394	5543

104 2.2 Positron annihilation spectroscopy

105 Thermalized positrons diffuse in the crystalline lattice and can move to a relatively low
106 electronic density position due to repulsion from the host nucleus. They eventually annihilate
107 with a surrounding electron, emitting two gamma rays of $511 \pm \Delta E$ keV. The energy deviation
108 ΔE depends essentially on the kinetic momentum of the annihilating electron. The gamma rays
109 from the annihilation, are collected in the measured DBS centered at 511 keV, representing the
110 momentum distribution of the annihilated positron-electron pairs starting at 0 m_0c (m_0 : electron
111 mass, and c : light speed). The low (resp. high) momentum parameter S (resp. W) corresponds
112 to the ratio of the counts in the low (resp. high) momentum region over the counts in the total
113 momentum range.

114 At CEMHTI Laboratory, a DB Spectrometer is coupled to a Slow Positron Beam (SPB-
115 DBS) that produces a monoenergetic positron beam with a variable energy ranging from 0.5 to
116 25 keV [47]. Annihilation gamma rays are recorded using a high-purity germanium detector
117 with an efficiency of over 25 % at 1.33 MeV. The spectrometer has a dead time of around 10 %
118 and an energy resolution of ~ 1.25 keV (full width at half maximum, FWHM), equivalent to a
119 momentum resolution of $4.89 \times 10^{-3} m_0c$. The S (resp. W) parameter is calculated in the
120 momentum range of $|2.64| \times 10^{-3} m_0c$ (resp. $(|9.80| - |24.88|) \times 10^{-3} m_0c$) of each DBS. According
121 to the Makhovian model [48], the maximum penetration of slow positrons in Mo is about 1300
122 nm. The VEPFIT program [49,50] is used to extract the depth distribution of defects from the
123 S and W values measured as functions of positron energy ($S(E)$ and $W(E)$) by modeling the
124 sample as several homogenous layers. Each layer has specific annihilation characteristics,

including the S , W parameters and an effective diffusion length of the positrons, i.e. L_{eff}^+ . The L_{eff}^+ is physically related to the concentration of defects which impede the positron diffusion and to the intrinsic diffusion coefficient of positrons D^+ , which is of about $1.2(1) \times 10^{-4} \text{ m}^2.\text{s}^{-1}$ at 300 K in Mo [51].

$$L_{eff}^+ = \sqrt{\frac{D^+}{\lambda_L + k_{tot}}} \quad (1)$$

Where λ_L is the defect-free Lattice annihilation rate, and k_{tot} the total trapping rate. λ_L is the inverse of the Lattice positron lifetime τ_L , which means the lifetime of positrons annihilating as delocalized in the perfect lattice. For Mo, τ_L was estimated experimentally and by calculation at values of 103 ps [52] and 106 ps [39], respectively. These values are close to those of tungsten of about 105 ps [41,52,53]. In a solid containing p types of vacancy defects (V_i), positrons can annihilate as delocalized in the perfect lattice and as being trapped in the different defects V_i , with a trapping rate k_i . The measured S and W parameters can be represented as a weighted sum of the specific S and W values associated with each type of annihilation state j (i.e. p defects and lattice) and their respective annihilation fractions (f_j), using a trapping model [36] with $p+1$ states. The f_i for individual potential defects i are given by the following expression:

$$f_i = \frac{k_i}{\lambda_L + k_{tot}} \quad (2)$$

The vacancy concentration C_i relates to the specific positron trapping coefficient μ_i and the trapping rate k_i , expressed as $C_i = k_i/\mu_i$. In W, the specific trapping coefficient of V_1 is about $(6 \pm 3) \times 10^{15} \text{ m}^3.\text{s}^{-1}$ [36,54] and should be equivalent in Mo due to similar crystalline structure and lattice parameters in both materials. For small vacancy clusters containing less than ten V_1 , their specific trapping coefficients are proportional to the number n of the included V_1 , i.e. $\mu_{V_n} = n \cdot \mu_{V_1}$ [36]. On the other hand, for the large vacancy clusters containing more than 10 V_1 , the estimation of specific trapping coefficients was detailed in [55].

2.3 First-principles calculations

The first-principles calculations were conducted by using the Vienna *Ab initio* Simulation Package (VASP) [56] with the projector augmented wave (PAW) method [57]. The generalized gradient approximation with the Perdew-Burke-Ernzerhof exchange-correlation function[58] was used for the electron exchange and correlation energy. The $6 \times 6 \times 6$ bcc supercells containing 432 lattice sites were used for all calculations. The plane-wave cutoff energy was set to 230 eV, and the Γ point only was used for all calculations. We chose the PAW potential, which treats the $4p^6 4d^5 5s^1$ electrons as valence states in Mo. The convergence criterion for the electronic loop was 10^{-5} eV. The force tolerance for ionic relaxation was 0.01 eV/Å. Hou et al. [59] have determined the stable structures of vacancy clusters by minimizing their Wigner-Seitz areas, the same structures of vacancy clusters were used in this study. Because for a give vacancy cluster in bcc metals, the configuration with minimum Wigner-Seitz area is always the same, regardless of the type of materials.

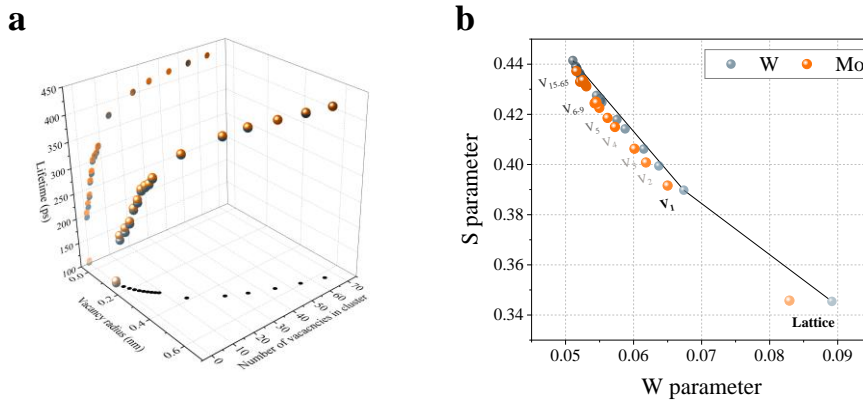
After the standard DFT calculations, the positronic structures were computed based on the electronic structures computed by two-component DFT [60]. The Rubaszek weighted-density approximation (WDA) [61] was used for the e^+e^- correlation potential to compute the positron densities. The positron implementation was developed and provided by Makkonen et al. [40], and the WDA part was developed by Callewaert et al. [62]. A detailed discussion about using WDA to compute the positron annihilation characteristics in vacancy clusters can be found in Ref. [41]. The positron was approximated not to affect the average electron density, and the zero-positron-density limit was used [60]. Although the zero-density-limit results in the non-self-consistent treatment of positron densities, it has been shown in Ref. [34] that the non-self-consistent calculation yields very similar results with the fully self-consistent calculations for small vacancy defects. The momentum distribution of annihilating e^+e^- pairs (Doppler spectra) was computed with the state-dependent scheme [63] and PAW method [40]. All computed

174 Doppler spectra were convoluted with a Gaussian function with a FWHM corresponding to the
 175 experimental resolution ($4.89 \times 10^{-3} \text{ m}_0c$).

3. Results

3.1 First-principles calculation

178 In our previous work [41], the two-component DFT was used to calculate the annihilation
 179 characteristics of several annihilation states in W. The evolution from the *Lattice* to V_1 and
 180 from V_1 to the largest vacancy cluster (V_N) was demonstrated in agreement with the experiment
 181 [41]. Accordingly, the two-component DFT was also employed to calculate *annihilation*
 182 *characteristics* (lifetime and S , W parameter) of various annihilation states (Lattice and
 183 vacancies) in Mo with identical parameters [41]. The line L_1 connects the *Lattice* and V_1 , S , W
 184 points corresponding to the annihilation between these two states with different fractions.



185 Fig. 1: two-component DFT-calculated a) positron annihilation lifetime and b) momentum parameters S vs W curves
 186 for Mo and W [41]; the vacancy radius was estimated assuming a spherical model.

187 Fig. 1 compares the positron annihilation characteristics of vacancy clusters in tungsten
 188 and molybdenum (positron lifetimes a and S - W plot b). Fig. 1a clearly shows that, with the
 189 increase of vacancy cluster size (from V_1 to V_{20}), the positron lifetime gradually increases from
 190 200 ps to 350 ps. Fig. 1b presents the S - W points of vacancy clusters in W and Mo. The S - W

points follow the same trend with the increase of vacancy cluster sizes. However, when the vacancy cluster size is greater than V_{27} (radius = 0.47 nm), the positron annihilation characteristics reach saturation, in this regime, the identifiable largest vacancy cluster is referred to V_N . When comparing the difference in the W parameter between Mo and W for the various annihilation states, the values for the **Lattice** are the most remarkable. The W is lower in Mo than in W, probably because the number of electrons close to the nuclei (core electrons) is higher for W. The S parameter which mainly reveals the annihilation of the valence electrons, is very close in both Mo and W. In addition, it should be noted that in the experimental spectra, the high momentum region has relatively low counts, making it challenging to highlight the difference in the W parameter, as shown in [supplementary II](#).

3.2 Positron annihilation spectroscopy

3.2.1 Reference sample

The $S(E)$ curve of the Mo reference sample annealed at 1200 °C for 2h is plotted in [Fig. 2a](#), the data was adequately fitted using VEPFIT in the one homogenous layer model. The annihilation characteristics were $S_{Ref} = 0.377$ (3), $W_{Ref} = 0.079$ (2), and the L_{eff}^+ of positrons equals 63 nm. The fitting ([Tab.1](#)) was carried out from 2 keV, as the annihilation fraction at the sample surface is important below this energy [41]. As the L_{eff}^+ is shorter than the intrinsic diffusion length (110 nm) determined at 300 K for Mo [48], this suggests that a small fraction of vacancy defects were not eliminated during the annealing process.

After a further annealing at higher temperature of about 1700 °C/2h under high vacuum (10^{-5} Pa), the L_{eff}^+ expands to 92 (18) nm, and the annihilation characteristics S_{1700} and W_{1700} (blue circle) overlap with the **Lattice** point in W (see the S - W curve in [Fig. 2b](#)), suggesting that the **Lattice** annihilation characteristics $S_L(\text{Mo})$ and $W_L(\text{Mo})$ for Mo are very close to those experimentally determined in W. This confirms the suggestion in the [part 3.1](#), i.e. the S , W parameters corresponding to **Lattice** and V_1 are very close for Mo and W.

216 In Fig. 2b, the S , W points extracted from $S(E)$ and $W(E)$ using VEPFIT for Mo samples
 217 are plotted with the Lattice, V_1 , and V_N points already determined for W [33,54,64]. The specific
 218 point (dark circle in Fig. 2b) of the reference Mo sample (W_{ref} , S_{ref}) is aligned on the V_1 line
 219 found in W and is located slightly at a higher left position on this line relative to the W **Lattice**
 220 point (red star in Fig. 2b). Furthermore, considering that defects are mostly V_1 , the residual
 221 defects concentration were calculated using equation (4) in literature [9] with the lifetime of the
 222 Mo Lattice of 103 ps [52]. The remaining vacancy concentration is about $3 \times 10^{24} \text{ m}^{-3}$ in the
 223 reference sample with a L_{eff}^+ of about 63 nm, and $5.5 \times 10^{23} \text{ m}^{-3}$ in the reannealed Mo sample
 224 with a L_{eff}^+ of 92 nm. It is important to note that, after the initial annealing at 1200 °C for 2
 225 hours, the remaining concentration of vacancy defects is too low to interfere with the study of
 226 irradiation-induced defects. This previous state before irradiation is named reference state i.e.
 227 *Ref* in the following discussion.

228 3.2.2 Self-irradiated samples

229 The $S(E)$ curves plotted in Fig. 2a, reveal the distribution of defects in Mo samples
 230 irradiated at fluence in the range from 2.1×10^{16} to $4.2 \times 10^{18} \text{ m}^{-2}$. The $S(E)$ curves have similar
 231 shapes for all fluences. S increases from the approximately identical value measured at 0.5 keV
 232 up to a maximum obtained as positron energy increases with fluence. Thereafter, S decreases
 233 towards, but does not reach, the value in the reference sample S_{Ref} . The S parameter discloses
 234 the size and/or concentration of open volume where e^+e^- pairs were annihilated. As the fluence
 235 increases, the S parameter increases, indicating the detection of a larger size and/or higher
 236 concentration of vacancy defects. However, the increase slows down when the fluence exceeds
 237 $6.3 \times 10^{17} \text{ m}^{-2}$. For the highly damaged region ($E < 10 \text{ keV}$), the $S(E)$ curves almost overlapped,
 238 and $W(E)$ curves exhibited a mirror evolution, suggesting that the defect distribution probably
 239 saturates. The $S(E)$ and $W(E)$ curves can be fitted using the VEPFIT program [15,16] by

describing the samples in three homogeneous layers. The fitted *annihilation characteristics* in Layers 1, and 2 are summarized in [Tab.2](#).

Tab. 2: Annihilation characteristics $S_{\text{Layer}(i)}$, $W_{\text{Layer}(i)}$, and $L_{\text{Layer}(i)}^+$ for layers $i = 1, 2$, extracted from the fitting of the $S(E)$ and $W(E)$ curves with the VEPFIT program using a three-layer model for samples irradiated at different fluences, ranging from 2.1×10^{16} to $4.2 \times 10^{18} \text{ m}^{-2}$. The corresponding peak damage value ranges from 0.01 to 2 dpa. R is the slope of the straight line connecting point $(S_{\text{layer}1}, W_{\text{layer}1})$ and the Lattice point in the S - W curve, i.e. $R = (S_{\text{layer}1} - S_L) / (W_{\text{layer}1} - W_L)$. The Layer boundary (L_b) is the depth from the sample surface at which each layer expands. The annihilation characteristics of Layer III are fixed to those of the reference sample ($S_{\text{layer}3} = 0.377$ (5), $W_{\text{layer}3} = 0.079$ (2), and $L_{\text{layer}3}^+ = 63$ nm). Surface annihilation characteristics are equivalent ($S_{\text{surf}} = 0.400$ (5), $W_{\text{surf}} = 0.065$ (3)) for all samples. The incertitude of the fitted value is about 0.004 and 0.002 for S and W , respectively.

Damage	Layer 1					Layer 2				
dpa	$S_{\text{layer}1}$	$W_{\text{layer}1}$	R	$L_{\text{layer}1}^{eff+}$	L_b (nm)	$S_{\text{layer}2}$	$W_{\text{layer}2}$	R	$L_{\text{layer}2}^{eff+}$	L_b (nm)
~0.01	0.421	0.057	2.05 (16)	14 (4)	200 (9)	0.399	0.069	2.08 (28)	50	716 (31)
~0.1	0.440	0.051	2.18 (10)	13 (2)	252 (9)	0.412	0.062	2.05 (15)	40	856 (33)
~0.3	0.448	0.049	2.25 (10)	12 (2)	227 (11)	0.424	0.057	2.08 (12)	35	865 (27)
~0.5	0.449	0.048	2.27 (9)	11 (2)	260 (13)	0.427	0.056	2.11 (12)	30	937 (27)
~1	0.451	0.047	2.27 (9)	10 (2)	311 (15)	0.427	0.056	2.07 (11)	30	945 (29)
~2	0.451	0.047	2.27 (9)	10 (2)	356 (19)	0.429	0.055	2.11 (11)	30	1000 (36)

[Fig. 2c](#) shows the evolution of the S parameter as a function of depth for each fluence. The first two layers displayed the damaged region, and the third one was fixed with annihilation characteristics of the reference sample (S_{ref} , W_{ref} , and $L_{\text{eff}}^+ = 63$ nm). In addition, the L_{eff}^+ in each layer is shown in [Fig. 2d](#) as a complement of the S parameter. The thickness of the first layer, L_b , and the effective positron diffusion L_{eff}^+ vary from 200 to around 356 nm, and from 10 to 17 nm, respectively. The fit confirms the damage profile predicted using the SRIM program, with the highest S and the shortest L_{eff}^+ found in Layer 1 for each sample. This suggests that most irradiation-induced defects are located in the first 300 nm. Thus, the fitted S , W value in Layer 1, i.e. $S_{\text{layer}1}$ as a function of $W_{\text{layer}1}$ were plotted in [Fig. 2b](#). The slope R of the straight line connecting each specific S - W point ($W_{\text{layer}1}$, $S_{\text{layer}1}$) to the **Lattice** point (W_L , S_L), calculated as the ratio $(S_{\text{layer}1} - S_L) / (W_{\text{layer}1} - W_L)$, reflects the evolution of the defect size.

At the lowest fluence ($2.1 \times 10^{16} \text{ m}^{-2}$), the width of Layer 1 is around 200 (9) nm. Compared to the reference sample, L_{eff}^+ has decreased by more than 30 %, from 63 nm to around 17 (2) nm, due to the positron trapping in irradiation-induced defects. In addition, the fitted values

263 $S_{lay1}=0.421(4)$, and $W_{lay1}=0.058(2)$ are close to those of V_1 experimentally determined in W
 264 (the green crossed circle in Fig. 2b), but slightly further to the right. The corresponding slope
 265 R is 2.05(16), which is slightly larger than that of the V_1 line in W (1.85(3) [9]). These results
 266 indicate that V_1 is a dominant trap for positrons, although it cannot be ruled out that a small
 267 fraction of positrons probably annihilate in vacancy clusters. In Layer 2, from 200 (12) to 716
 268 (31) nm, S_{lay2} is lower than S_{lay1} , and the L_{eff}^+ is about 50 nm. Although the irradiations still
 269 induced defects in this layer, their concentration is considerably lower than in Layer 1, and
 270 annihilation in the **Lattice** becomes significant as the L_{eff}^+ lengthens.

271 The S and W parameters of the vacancy clusters (V_2 - V_{55}) in W were also plotted in Fig. 2b.
 272 Their values were transposed in the experimental frame network from theoretical values using
 273 the experimental and theoretical V_1 - V_N lines as the references [55]. As the fluence increases,
 274 the fitted annihilation characteristics in Layer 1 vary following defect size evolution. Once the
 275 peak damage dose has reached around 0.5 dpa, the thickness of the Layer 1 extends to about
 276 260 nm, and the variation of annihilation characteristics is discontinued (Tab.1). The S (resp. W)
 277 parameter converges around 0.451 (resp. 0.047). The R ratio reaches the value 2.27 (9), and the
 278 S , W points are located between the S , W points corresponding to the clusters V_3 and V_4 (Fig.
 279 2c), revealing the formation of small vacancy clusters. At the same time, the L_{eff}^+ shortened to
 280 about 10 nm (Fig. 2d), showing the increase of the positron trapping rate. For a damage dose
 281 greater than 0.5 dpa, the S - W points for Layer 1 remain almost superimposed, indicating that
 282 the defect distribution reaches a steady state in the first 350 nm in the sample. A similar
 283 saturation effect was already observed in W self-ion irradiated at RT [9,11]. The saturation
 284 point for W is plotted in the cyan sphere in Fig. 2c [9]. Remarkably, the saturation S (resp. W)
 285 parameter is higher (resp. lower) in Mo than in W, suggesting that the proportion of the vacancy
 286 clusters is higher in Mo than in W.

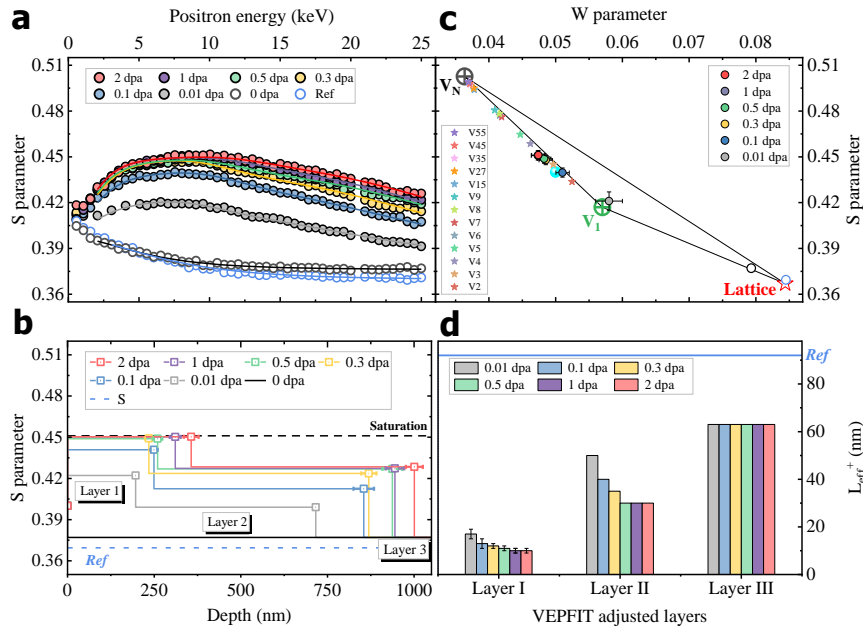


Fig. 2: Annihilation characteristics in Mo samples before and after self-irradiation: a) S parameter as a function of positron energy E , b) depth profile of parameter S extracted for the first layer from the $S(E)$ and $W(E)$ using VEPFIT; c) S - W plot of the S and W extracted for the first layer in the reference annihilation characteristics of Lattice (open red star), single vacancy (green crossed-circle) and saturation signal V_N (dark crossed circle) determined in W [33,35,64], the colored solid stars represent the estimated S, W for various vacancy clusters. The cyan point corresponds to the steady-state obtained in self-irradiated W [9] d) positron effective diffusion length (L_{eff}^+) in adjusted layers using VEPFIT program.

In addition, deeper damage was created with increasing fluence (as shown in Fig. 2c), extending much deeper than the 300 nm depth predicted by the SRIM simulation for Mo irradiated with 1 MeV self-ions [8]. A similar result has been observed by He et al.[65] in the case of implantation of yttrium, titanium, and oxygen in Fe. This difference is due to the channeling effect in the polycrystalline sample with large grain size ($> 50 \mu\text{m}$), which is not considered in the SRIM calculations.

4. Discussion

4.1 Quantitative evaluation of defect size distribution

Recently, we extracted complete vacancy distributions in self-ion irradiated W, using a quadratic programming and simulated annealing algorithm (SA)[42], parametrized with a

304 positron trapping model including pure vacancy defects V_i with $i=1-65$ [55]. The concentration
305 and proportion of each vacancy defect were estimated using the same method in the self-
306 irradiated Mo samples. The results are shown in 3a and b, where clusters containing more than
307 15 vacancies are not displayed, as their concentration is close to the lower detection limit (Fig.
308 7 in Supplementary Information) of SPB-DB. For the lowest damage level (0.01 dpa), the
309 estimated proportion of V_1 is 99%, and the concentration reached around $5 \times 10^{25} \text{ m}^{-3}$. This is
310 in line with the first conclusion made in the qualitative interpretation of the experimental results,
311 which established that V_1 is the dominant trap for positrons with a minor proportion of vacancy
312 clusters at the lowest fluence (0.01 dpa).

313 When the damage escalates from 0.01 to 0.1 dpa, the proportion of V_1 decreases to 48%,
314 and ~40% vacancy clusters account for V_{2-3} , and ~10% for V_{4-7} . Up to the 0.3 - 0.5 dpa, V_1 and
315 V_{4-6} clusters represent a quarter of the total proportion. Following the damage accumulation
316 until 2 dpa, the concentration of V_1 declined again, and its proportion reduced to about 20 %.
317 At the same time, the concentration and proportion of V_{4-9} manifestly increased. According to
318 DFT, the V_1 is less mobile at RT, and binding energy is predicted to be negative for V_2 ,
319 whereas the vacancy clusters can still agglomerate in the cascades or the overlapped cascades
320 resulting in large clusters. The larger cluster of 15 vacancies (radius = 0.35 nm) reached the
321 lower limit of the SPB-DB ($\sim 3 \times 10^{22} \text{ m}^{-3}$, Fig. 3a and Fig.8 in the Supplementary information)
322 as the damage level increases) to 0.5 dpa. In addition, each size of vacancy reached a stable
323 level when the damage accumulated at 0.5 dpa or more, agrees with the computational work in
324 case of tungsten [66].

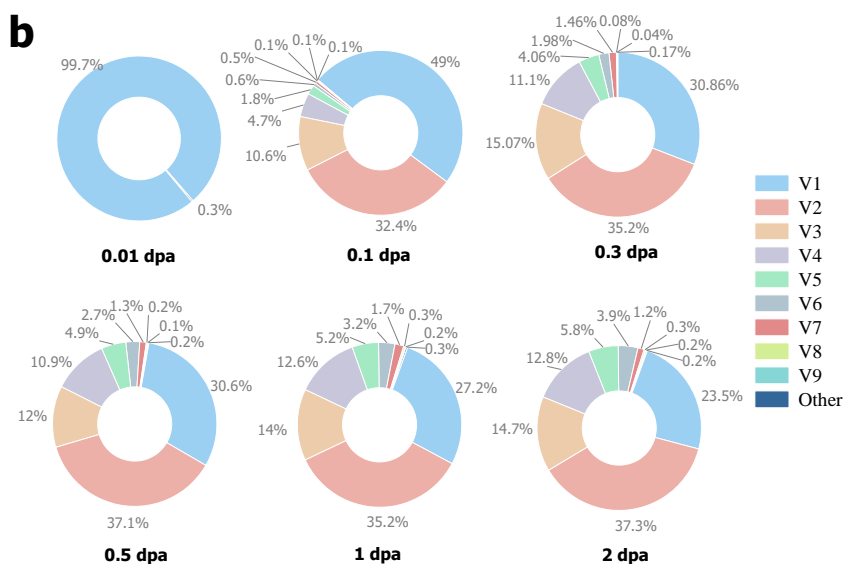
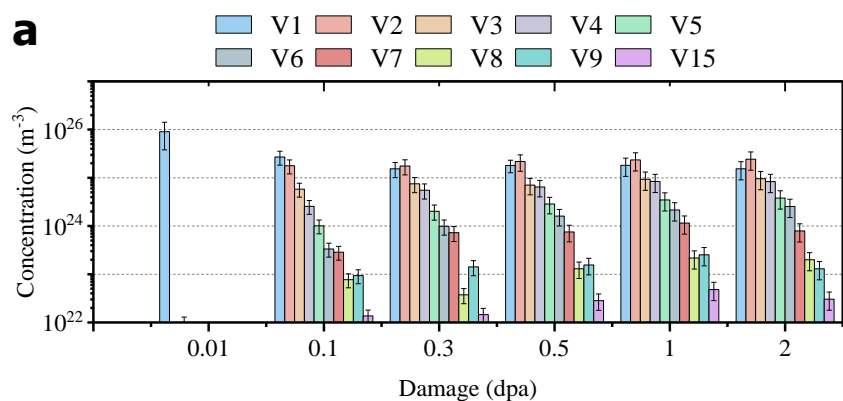


Fig. 3: a) concentration and b) proportion of the vacancy defects probed by positrons in 1MeV self-ion, RT-irradiated Mo in the first ~ 300 nm below the surface (corresponding to the first layer) for damage levels ranging from 0.01 to 2 dpa (with TDE= 34 eV, SRIM2008-K-P). These values were extracted from the S and W experimental values using the methodology described in [42]

Selby et al. [67] operated molecular dynamics (MD) to mimic the evolution of the defects at 300 K in Mo. The simulation involved Primary Knock-on Atom (PKA) energies up to 50 keV, whereas 98% PKA have an energy higher than 50 keV according to SRIM with experimental conditions. These simulated results showed over 80 % of vacancies were isolated

single vacancies, and the largest vacancy cluster contained nine vacancies. This result might compare with low fluence experience, in cases where the cascades overlap is less pronounced.

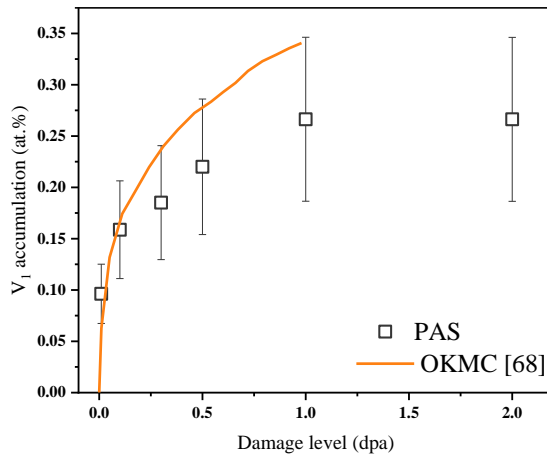


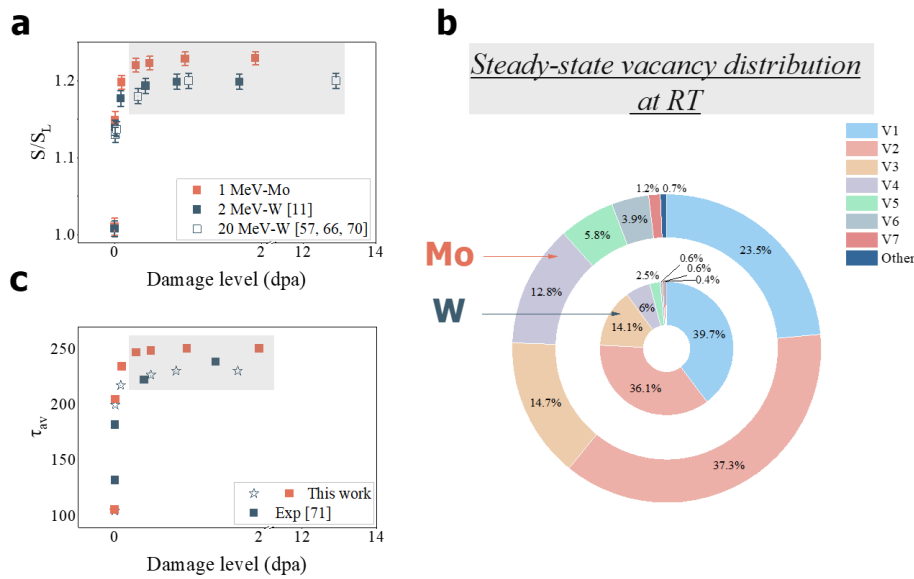
Fig. 4: comparison of estimated V_1 accumulation as a function of the damage level using PAS and OKMC [43], the computational value averages the vacancy concentration value in the first 400 nm, and the experimental data represent the vacancy accumulation in Layer I obtained from the fit, its thickness varies with damage level from 200-356 nm smuorized in Table.2.

Recently, Hou et al. [68] employed OKMC simulation under our irradiation experiments to better align with MD on the damage accumulation time scale with the present experiment. The authors showed that the SIA defects had annihilated quickly with vacancies or at the sample surface. Despite the limited diffusion of vacancies at 300 K, vacancy clusters ($> V_6$) formed. The authors suggest that such vacancy clusters may form within the core of the cascade due to the high density of vacancies, enabling short-scale agglomeration after diffusion or as a result of cascade overlapping. These results are consistent with the estimated vacancy clusters proportion derived from the PAS data in this study, as illustrated in Fig. 3. Furthermore, the authors observed vacancy depletion at depth beyond 400 nm, correlating well with the decrease in the S parameter shown in Fig.2 (a-c). Fig.4 compares the vacancy accumulation (total concentration of isolated V_1 and those included in the vacancy clusters) as damage increases, based on PAS data and OKMC simulations. Both computational and experimental results reveal

354 similar trends. In addition, the experimental data reveal that V_1 accumulation reaches saturation
 355 when the damage level exceeds 0.5 dpa.

356 4.2 Steady-state vacancy distribution

357 Focusing on the vacancy steady state, Fig.5a compares the relative S parameter, S/S_L
 358 (relative S parameter to Lattice), as it varies with damage accumulation in Mo in the present
 359 study and previously reported data in W [9,54,64,69]. A steady-state behavior is observed in
 360 both materials, though Mo exhibits a higher S saturation value. Vacancies in both metals
 361 accumulate rapidly up to 0.5 dpa, after which their proportion converges. Notably, defect
 362 evolution appears to be independent of self-ion energy, as similar trends are observed for 2 and
 363 20 MeV self-ion irradiation in W [9,54,64,69].



364
 365 Fig. 5: a) the relative S value, S/S_L , and b) the proportion of aggregated vacancies as a function of the damage level caused by
 366 self-ion in Mo (1 MeV) and W (2 MeV & 20 MeV) at RT, c) The e^+ annihilation lifetime estimated (hollow star) for each damage
 367 levels in comparison with experiments [70]

Fig. 5b highlights that, at the defect steady state (> 0.5 dpa), the proportion of vacancies within clusters is significantly higher in Mo (76.5%) compared to W (60%). To further affirm the difference in vacancy proportions between Mo and W, we calculated the averaged lifetime τ_{av} by weighing the lifetime of vacancies (using data in Fig. 1a) with the annihilation fractions determined by SA [55]. Fig. 55c shows the progression of τ_{av} with damage level. For W, the estimated lifetime evolution with damage accumulation coincides with experiments [70], validating the SA method for estimating annihilation fractions. A lifetime of around 231 ps was found for the steady state of defects in W. In comparison, a longer τ_{av} of 250 ps is determined for Mo, consistent with the SPB-DBS results, indicating a higher proportion of large vacancies in Mo since the positron lifetime for each type of vacancy defect are very close in both metals Mo and W (see Fig 7, and Table3 in Supplementary Materials). It is also noteworthy that the binding energy of V_{2-3} is relatively close to zero for W and slightly negative for Mo, according to references [71–73].

The vacancy size distribution could be strongly affected by the binding and migration of vacancy defects and interstitials. We calculated the binding energies for V_1 to V_9 using DFT calculations (see Fig. S1 in the SI). Notably, the binding energy of V_2 - V_3 is near zero for W and slightly negative for Mo, as reported in Refs. [71–73] at 0K. To date, whether V_2 - V_3 forms at RT in Mo or W remains unresolved. Only Park et al.[74] reported a binding energy of about 0.7 eV for V_2 in W using Field-ion spectroscopy. Impurity atoms are also known to stabilize V_2 [75,76]. Recently, Zhang et al.[77] estimated the effect of temperature on the binding free energy, finding that the latter increases with rising temperature. Although V_2 remains unstable at RT, two isolated V_1 s located at the first nearest-neighbors tend to bind with each other from 700 K, which is an attainable temperature during cascades. Given this, we considered the possibility of the e^+e^- annihilations in V_2 and V_3 . Moreover, it is remarkable that although the cluster containing less than seven V_1 account for 99 % of the total vacancy proportion in both

metals, larger vacancies may also be formed, though in a limited concentration. Quantification of these larger defects is difficult, as for cluster with more than 15 V_1 fall outside the optimal detection range for the SPB-DBS method, as shown in Fig. 8 in Supplementary I.

The migration energy E^m is the lowest for the self-interstitial atoms (SIA) (Fig.1 in the supplementary). For one-dimensional diffusion in a $\langle 111 \rangle$ direction, E_{SIA}^m is about 0.064 eV in Mo and 0.040 eV in W [78]. Thus, the SIA should be very mobile due to their very low migration energy in the Mo and W matrix. For the small vacancies, the tri-vacancy has the lowest E^m in both materials, and for the other vacancy defects V_1 , V_2 , and V_4 , their E^m is lower for the Mo matrix. Hou et al.[43] calculated using DFT, the binding energy for larger clusters containing up to 47 vacancies in Mo. They showed that E^m increases more or less with the number of vacancies in the cluster up to 15, reaching 2 eV and then remaining between 1 and 2 eV. It has to be noted that the probability of dissociation of vacancy clusters is very low at RT. Agglomeration is possible if vacancies can diffuse and link up with each other. Based on the Arrhenius diffusion equation, the distances that V_1 and V_3 can travel at 300 K during the duration of the irradiation at 0.5 dpa (2100 s) are estimated to be about 2×10^{-5} nm and 7.8×10^{-3} nm, respectively. However, the local annealing due to the collision of the ion might enhance their mobility, so the agglomeration of vacancies might occur in the core of cascades, where the density of vacancies is very high, facilitating short-scale diffusion and interactions among vacancies. In addition, vacancy diffusion and binding should be more probable in Mo, given its lower migration energy compared to W. The higher proportion of vacancies within clusters in Mo could, in part, be attributed to the difference in their evolution mechanisms.

On the other hand, De Baker et al. [79] determined the fragmentation energy¹ (E_{fr}) in various materials using MD and Binary Collision Approximation (BCA) simulations. They

¹ The energy above which one cascade can split into several subcascades

found that the number of subcascades increased linearly with the PKA energy when E_{PKA} exceeds E_{fr} , determined to be 17 keV for Mo and 75 keV for W. In bcc metals, cascades overlapping were found to consistently reduce the size of the pre-existing large voids (>100 vacancies) when cascades partially overlap with the defect in W [80] and Fe [13]. In Fe, the size of all clusters -except for the single vacancy- was reduced after a fully overlapping cascade. This reduction depended on PKA energy and interatomic potential but was independent of the initial cluster size [81].

Furthermore, the recombination of FPs in both Mo and W is similar, primarily due to the high mobility of the SIAs. Consequently, the recombination rate of FPs is expected to be comparable in Mo and W, given the equivalent migration energy of SIAs (Fig.1 in the supplementary). However, the TDE in Mo is lower than in W. The arc-dpa model [76] was used to quantify these differences to estimate the average number of FPs produced per cascade. The $b_{arc-dpa}$ and $c_{arc-dpa}$ parameters were taken from Ref. [82]. For Mo, $b_{arc-dpa}$ value was taken from Ref. [83], in which it was suggested that -1 is a proper value for most metals. The $c_{arc-dpa}$ parameter represents the cascade efficiency. According to references [67,84], using classical MD, the cascade efficiency of pure Mo is ~0.2. Therefore, $c_{arc-dpa} = 0.2$ was used. Using the TDE value in Table 4, the arc-dpa model estimated the average number of defects produced in a single cascade event. The results indicate a slightly higher number of created defects in Mo (640 FPs) compared to W (559 FPs).

Tab. 3: related energies to the evolution of defects in W and Mo, E_{fr} : fragmentation energy, over which the cascades could split into several subcascades, TDE: threshold displacement energy, energies are presented in keV, mean: mean PKA energy, equal approximately to a half of the maximum value in the PKA energy predicted by SRIM program. T_d : damage energy kinetic energy available to provoke atomic displacement after losing the electronic ionization energy, kinetic energy available to provoke atomic displacement after removing the electronic ionization energy.

	E_{fr} (keV)	TDE (eV)	T_d (keV)	Number of FPs created in one cascade
Mo	17 ^a	34 ^b	282 ^d	640 ^d
W	75 ^a	55 ^c	600 ^d	559 ^d

^a reference [79], ^b reference [85], ^c reference [86], ^d reference [87].

441 The number of vacancies created by a single event of cascades is higher in Mo than in W
442 using the arc-dpa model. After some of these vacancies recombine with SIAs, the remaining
443 vacancies in Mo have a stronger clustering tendency than those in W. In addition, it appears
444 that vacancies in W are less mobile at RT, as indicated by the migration energy presented in
445 Fig.1 in the supplementary. The higher proportion of vacancy clusters in Mo compared to W is
446 likely due to subtle differences in collision cascade overlapping and Frenkel pair recombination
447 between the two materials, resulting in a difference from the early stage of the evolution.

448 Conclusion

449 In this study, we combined the PAS, first-principles calculation, and SA algorithm to
450 quantitatively analyze the size distribution of vacancy defects induced by self-ion in Mo and W
451 at RT. The proportion of each vacancy defect from V_1 to V_9 was depicted for the first time
452 based on experimental data, and the primary conclusions have been reached as follows:

- 453 (i) Comparable e^+ annihilation characteristics in vacancy clusters in W and Mo: The
454 annihilation characteristics exhibit similar evolution from V_1 to the saturation (V_{35} ,
455 ~ 0.5 nm in radius).
- 456 (ii) At low irradiation dose (~ 0.01 dpa), the predominant defects observed are nearly 100%
457 V_1 in both Mo and W. At 0.3 dpa, the presence of V_{15} (radius = 0.35 nm) becomes
458 detectable by PAS, and the vacancy aggregation rate converges with a damage level
459 of about 0.5 dpa or more. The vacancy accumulation exhibits a trend comparable to
460 that of the OKMC simulation. Although the damaged zone detected using SPB-DBS
461 is deeper than those OKMC or SRIM estimated, probably due to the channeling
462 effect.
- 463 (iii) At the defect steady state (> 0.5 dpa) in Mo, the proportion of V_1 decreases to
464 approximately 20 %. The proportion of V_1 accounted for about 40 % of total
465 vacancies in W, whereas accounting for 23.5 % in Mo, revealing a higher vacancy

clustering tendency in Mo. Each size of the vacancy fraction reached a stable level as predicted by computational work.

These findings will significantly enhance the investigation of vacancy defects in metals. The method enables reliable quantitative analysis of vacancy proportions from primary defect distribution until the steady state at RT, addressing the scarcity of experimental data.

Declaration of Competing Interest

The authors declare that they have no known competing financial interests or personal relationships that could have appeared to influence the work reported in this paper.

Credit authorship contribution statement

Zhiwei Hu: PAS experiments, data processing and interpretation, writing – original draft & editing, **Qigui Yang**: First-principles calculation, data analysis, writing – original draft & editing, **Jingtong Wu**: data processing, interpretation, **Duc Nguyen-Manh & M. Yu. Lavrentiev**: Irradiation experiments **Pierre Desgardin**: Supervision, SPB-DBS experiments **Jérôme Joseph**: Sample preparation and technical support, **Marie-France Barthe**: Conceptualization, validation, data processing and interpretation, writing – review & editing.

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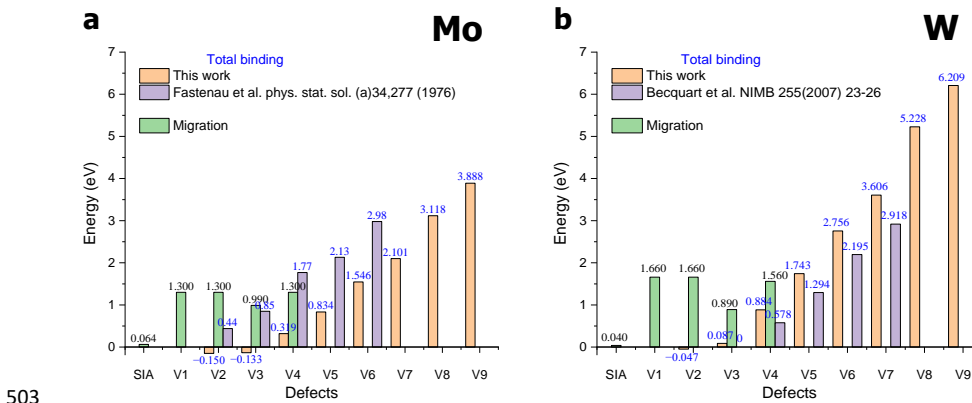
491 Supplementary

492 I. Binding and migration energy of vacancy clusters in Mo and 493 W

494 Table. 1 summarizes the binding energy E_b of a cluster consisting of n vacancies, V_n , which are
 495 calculated as follows :

$$496 \quad E_b = nE_{V_1} - E_{V_n} - (n-1)E_{\text{lattice}} \quad (1)$$

497 in which E_{V_1} and E_{V_n} are the energies of supercells with a mono-vacancy and one V_n cluster,
 498 respectively. E_{lattice} is the energy of a supercell of perfect lattice. Using the Eq. (1), positive
 499 binding energy means attractive interaction and negative binding energy means the opposite.
 500 The total binding energy increases with increase number of the V_1 including in the cluster,
 501 similar results are found in the Ref. for Mo[88] and for W[73]. The migration energy for
 502 vacancy defects from V_1 to V_4 is taken from the references [77-79].



504 Figure 6: Reported migration energy from V_1 to V_4 in references [77-79], and binding energy of various vacancies with
 505 relaxation calculated using DFT in the present work, compared with literature in a) Mo [88] and b) W[73].

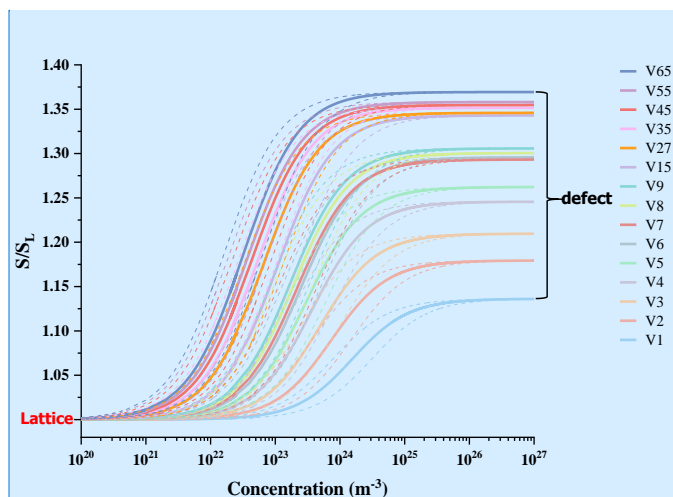
II. Sensitivity of the SPB-DBS

After calculating the specific trapping of vacancies, μ_{Vi} as it is explained in [36], we applied a one-trap trapping model for each vacancy. S parameter can be expressed versus defect concentration C_{Vi} by equation (4):

$$S = \frac{C_{Vi}\mu_{Vi}S_{Vi} + \lambda_L S_L}{C_{Vi}\mu_{Vi} + \lambda_L} \quad (4)$$

where S_{Vi} is the parameter for the vacancy defect Vi as calculated using TCDFT.

Fig. 7 presents the sensitivity of the SPB-DBS to the vacancy concentration in Mo for various vacancy defects from V_1 to V_{65} . When the vacancy concentration is 0, the annihilation fully occurs at the **Lattice**, the relative S value, S/S_L , equals 1. When the positrons are trapped in vacancies, the S/S_L increases with the vacancy concentration until a specific value which depends on their size. From V_{15} (radius = 0.35 nm), the positron annihilation characteristics start to converge, with a concentration higher to 10^{26} m^{-3} . The corresponding S/S_L value is close to 1.35. In the case of V_1 (radius = 0.139 nm), the sensitive range of concentration is between about $5 \times 10^{23} \text{ m}^{-3}$ and $8 \times 10^{23} \text{ m}^{-3}$, and the optimal resolution ranges from $3 \times 10^{23} \text{ m}^{-3}$ to $3 \times 10^{25} \text{ m}^{-3}$, and the S/S_L equals 1.136 also equals to S_{Vi}/S_L . For the vacancy size superior to V_{27} (radius = 0.47 nm), its lower detection limit is about $5 \times 10^{26} \text{ m}^{-3}$ and upper limit delciens to $8 \times 10^{23} \text{ m}^{-3}$, respectively. The most sensitive concentration range is between 5×10^{21} and $5 \times 10^{23} \text{ m}^{-3}$, and the S/S_L attains 1.32.



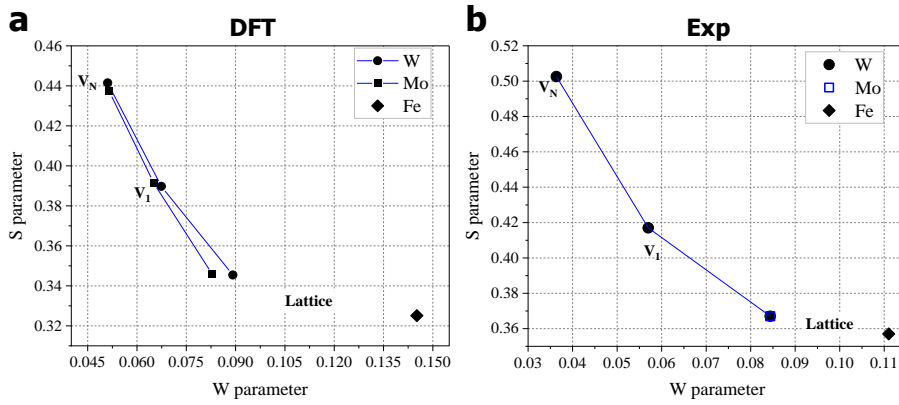
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Fig. 7: Sensitivity of PAS for each vacancy cluster in Mo using a one-trap trapping model, to be safe, the error bar (shown in dashed line) on the concentrations was set from the uncertainty of the specific trapping coefficient from [55]

Table.3 compares the calculated positron lifetime in various annihilation states from Lattice to a cluster containing 65 V1s. For each state, the lifetime is analogous, particularly when a cluster contains more than 8 V1s.

Table 3: Calculated positron annihilation lifetime in the lattice and vacancies in W and Mo using LDA

Annihilation state	Positron Lifetime (ps)	
	W	Mo
Lattice	101	106
V1	195	205
V2	214	223
V3	233	238
V4	261	265
V5	276	282
V6	299	303
V7	306	310
V8	312	314
V9	323	325
V15	374	375
V27	402	403
V35	413	414
V45	420	420
V55	424	425
V65	428	428



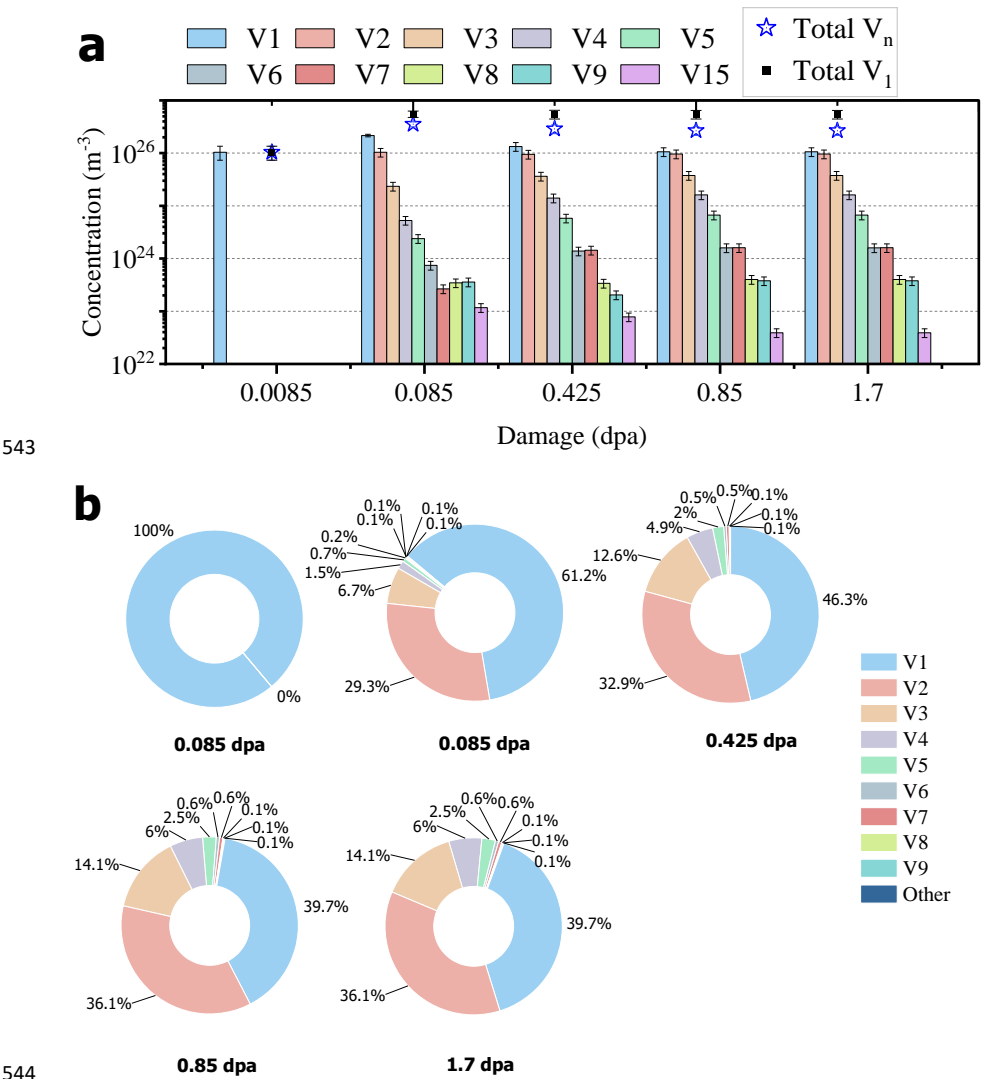
531 Fig. 8: a) Comparison of theoretical S - W values of Lattice, V_I , and V_N for W, Mo, and Fe calculated using two-component DFT
 532 [41], b) experimental annihilation characteristics of Lattice in W and Fe [89], and V_I and V_N in W [33,64].

533 Fig.9 compares the lattice annihilation characteristics of W, Mo, and Fe, using the values
 534 of S and W of V_I , V_N , and lattice determined in W as reference coordinates. The experimental
 535 results show that the difference on the annihilation characteristics of the *Lattice* of Fe and W
 536 are less pronounced than theoretical values. And it is then consider that the S , W of Mo and W
 537 is not disinguishable.

538

539 **III. Proportion of vacancies in self-ion irradiated W**

540 **Fig. 9** represents the vacancy proportion in self-ion irradiated W with a scope of damage
541 levels ranging from 0.0085 dpa to 1.7 dpa [9]. The V_1 was also the dominant defect at the lowest
542 damage level.



544 **Fig. 9:** a) concentration and b) proportion of the vacancy defects probed by positrons in 2 MeV self-ion [9], RT-irradiated W at
545 first ~ 300 nm for damage levels ranging from 0.0085 to 1.7 dpa (with TDE= 55 eV, SRIM2008-K-P)
546

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